



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 1, 2013

All Power Reactor Licensees and
Holders of Construction Permits in
Active or Deferred Status

SUBJECT: SUPPLEMENTAL INFORMATION RELATED TO REQUEST FOR INFORMATION PURSUANT TO TITLE 10 OF THE *CODE OF FEDERAL REGULATIONS* 50.54(f) REGARDING FLOODING HAZARD REEVALUATIONS FOR RECOMMENDATION 2.1 OF THE NEAR-TERM TASK FORCE REVIEW OF INSIGHTS FROM THE FUKUSHIMA DAI-ICHI ACCIDENT

On March 12, 2012, the U.S. Nuclear Regulatory Commission (NRC) issued a request for information pursuant to Title 10 of the *Code of Federal Regulations*, Part 50 (10 CFR), Section 50.54(f). The purpose of that request was to gather information concerning, in part, the external flooding hazards at operating reactor sites to enable the NRC staff to determine whether your license should be modified, suspended, or revoked. The "Required Response" section of Enclosure 2 indicated that licensees should provide the Hazard Reevaluation Report within 1 to 3 years from the date of the letter depending on their categorization.¹ Licensees whose current design basis does not bound the reevaluated flooding hazard were also requested, within 2 years of submitting the Hazard Reevaluation Report, to provide an Integrated Assessment Report that evaluates the total plant response to the reevaluated hazard. However, the results of the Hazard Reevaluation Report, in and of itself, are unlikely to provide sufficient information to determine whether there is a potential safety issue at the site. Nevertheless, those licensees performing an integrated assessment were requested to provide a list of any interim actions, taken or planned, to address the reevaluated hazard while the longer-term analysis takes place.

During the course of public interactions, various questions were raised with respect to the submission of the Hazard Reevaluation Reports (i.e., operability and reportability, interim actions, and extension requests). These issues were specifically discussed in a public meeting on February 11, 2013.² The purpose of this letter is to provide you with the NRC staff position regarding those issues discussed at the public meeting.

Operability and Reportability

Multiple questions regarding the impact of the Hazard Reevaluation Report on Structure, System, and Component (SSC) operability and reportability have been received in the course of stakeholder interaction. As with any new information that may potentially impact a nuclear power plant, licensees should consider and disposition the information through their corrective action program or equivalent process.

The staff considers the flood hazard reevaluations being performed pursuant to the 50.54(f) letter to be beyond the current design/licensing basis of operating plants. Consequently, the results of the analysis performed using present-day regulatory guidance, methodologies, and information would not generally be expected to call into question the operability or functionality

¹ The categorization of sites is available in the Agencywide Documents Access and Management System (ADAMS) under Accession no. ML12097A509.

² The February 11, 2013, meeting summary is available in the ADAMS under Accession No. ML13045B074.

of SSCs. Therefore, the results are not expected to be reportable pursuant to 10 CFR 50.72, "Immediate notification requirements for operating nuclear power reactors," and 10 CFR 50.73, "Licensee event report system." However, as with any new information that may arise at a plant, licensees are responsible for evaluating and making determinations related to operability and any associated reportability on a case-by-case basis.

It should be noted that the NRC staff does consider that errors identified in the current design/licensing basis during the performance of the requested flood hazard evaluation would necessitate an assessment of the operability of the affected SSCs. Additionally, this situation is likely to be reportable pursuant to 10 CFR 50.72 and 50.73. Licensees would also be expected to determine whether aspects of 10 CFR 50.9, concerning the requirement to provide complete and accurate information to the NRC, would be applicable.

Notwithstanding the preceding discussion, and as noted in the 50.54(f) letter, based upon the results of the review of the responses and other available information, the staff may impose additional requirements to protect against the reevaluated flood hazard. As always, the safety of the operating plants is of paramount importance. The NRC staff will follow established regulatory processes, including the backfit rule, in determining whether additional requirements are warranted. Further, as with any submittal to the NRC, licensees should evaluate the content to determine if it requires special treatment (e.g., security-related, proprietary, etc.) and request the information be withheld from public disclosure, as appropriate.

Interim Actions (Requested Information Item 1.d)

Licensees whose Hazard Reevaluation results are not bounded by their current design basis were requested to describe in their 50.54(f) letter response interim actions, taken or planned, to address the reevaluated flooding hazard while the staff assesses the safety and regulatory significance of the reevaluated hazard. The staff's review of the proposed interim actions will leverage appropriate sections and concepts from existing guidance documents such as NEI 12-07, "Guidelines for Performing Verification Walkdowns of Plant Flood Protection Features,"³ and JLD-ISG-2012-05, "Guidance for Performing the Integrated Assessment for External Flooding,"⁴ to evaluate the acceptability of the interim actions.

Licensees should describe the interim actions in sufficient detail to allow the NRC staff to assess their acceptability, in order to allow licensees the time needed to perform the integrated assessment and then implement permanent plant modifications, if necessary. The NRC staff will consider the appropriateness of the interim actions in the context of a licensee's ability to respond to the reevaluated flooding hazard(s) and how these actions continue to provide assurance of the licensee's ability to maintain the plant in a safe condition.

Extension Requests

In the event that a licensee is unable to submit a complete Hazard Reevaluation Report by its due date, the licensee should instead submit an extension request on or before that date. As discussed in the February 11, 2013, public meeting, incomplete hazard reports that only contain an analysis of some flooding hazard mechanisms would not be of substantive benefit for staff

³ The NRC endorsement of NEI 12-07 is available under ADAMS Accession No. ML12144A142.

⁴ JLD-ISG_2012-05 is available under ADAMS Accession No. ML12311A214.

review and would not be acceptable. Therefore, licensees should not submit partial reports. An extension request should include: 1) the reason for the delay; 2) a proposed schedule for the submittal of a complete Hazard Reevaluation Report; and 3) the basis for the acceptability of the revised schedule. The NRC staff will review schedule extension requests on a case-by-case basis to determine whether a request should be granted.

If you have any questions on this matter, please contact your NRC licensing Project Manager.

Sincerely,

A handwritten signature in black ink, appearing to read "Eric J. Leeds". The signature is fluid and cursive, with a large initial "E" and "L".

Eric J. Leeds, Director
Office of Nuclear Reactor Regulation

Enclosure:
List of Addressees

cc: Listserv

POWER REACTOR LICENSEES AND HOLDERS OF
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License Nos. DPR-51 and NPF-6

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License Nos. DPR-66 and NPF-73

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Bellefonte Nuclear Power Station

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Construction Permit Nos. CPPR-122 and
CPPR-123

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Braidwood Station

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License Nos. NPF-72 and NPF-77

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Browns Ferry Nuclear Plant

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License Nos. DPR-33, DPR-52 and DPR-68

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License Nos. NPF-11 and NPF-18

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License Nos. DPR-38, DPR-47 and DPR-55

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