



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 28, 2012

Mr. John F. McCann
Director, Nuclear Safety and Licensing
Entergy Operations, Inc.
440 Hamilton Avenue
White Plains, NY 10601

SUBJECT: ARKANSAS NUCLEAR ONE, UNIT NOS. 1 AND 2; BIG ROCK POINT PLANT; JAMES A. FITZPATRICK NUCLEAR POWER PLANT; GRAND GULF NUCLEAR STATION, UNIT 1; INDIAN POINT NUCLEAR GENERATING UNIT NOS. 1, 2, AND 3; PALISADES NUCLEAR PLANT; PILGRIM NUCLEAR POWER STATION; RIVER BEND STATION, UNIT 1; VERMONT YANKEE NUCLEAR POWER STATION; AND WATERFORD STEAM ELECTRIC STATION, UNIT 3 - ISSUANCE OF AMENDMENTS RE: REQUEST FOR APPROVAL OF CHANGE TO THE ENTERGY QUALITY ASSURANCE PROGRAM MANUAL AND ASSOCIATED PLANT TECHNICAL SPECIFICATIONS REGARDING STAFF QUALIFICATIONS (TAC NOS. ME774 TO ME7786)

Dear Mr. McCann:

The Nuclear Regulatory Commission has issued amendments consisting of changes to the Technical Specifications (TSs) in response to your application dated December 13, 2011, as supplemented by letters dated May 21 and November 20, 2012. The following amendments are enclosed:

- Amendment Nos. 248 and 296 to Renewed Facility Operating License Nos. DPR-51 and NPF-6 for Arkansas Nuclear One, Unit Nos. 1 and 2;
- Amendment No. 304 to Renewed Facility Operating License No. DPR-59 for James A. FitzPatrick Nuclear Power Plant;
- Amendment No. 193 to Facility Operating License No. NPF-29 for Grand Gulf Nuclear Station, Unit 1;
- Amendment Nos. 271 and 248 to Facility Operating License Nos. DPR-26 and DPR-64 for Indian Point Nuclear Generating Unit Nos. 2 and 3;
- Amendment No. 249 to Renewed Facility Operating License No. DPR-20 for Palisades Nuclear Plant;
- Amendment No. 239 to Renewed Facility Operating License No. DPR-35 for Pilgrim Nuclear Power Station;

- Amendment No. 178 to Facility Operating License No. NPF-47 for River Bend Station, Unit 1;
- Amendment No. 253 to Renewed Facility Operating License DPR-28 for Vermont Yankee Nuclear Power Station; and
- Amendment No. 240 to Facility Operating License No. NPF-38 for Waterford Steam Electric Station, Unit 3.

The amendment changes the Entergy Quality Assurance Program Manual and Associated Plant TSs regarding staff qualifications. The changes standardize unit staff qualification requirements for the Entergy fleet.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,



N. Kalyanam, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-313, 50-368, 72-13
50-155, 72-43, 50-333, 72-12, 50-416,
72-50, 50-003, 50-247, 50-286, 72-51,
50-255, 72-7, 50-293, 50-458, 72-49,
50-271, 72-59, 50-382, and 72-75

Enclosures:

1. Amendment No. 248 to DPR-51
2. Amendment No. 296 to NPF-6
3. Amendment No. 304 to NPF-59
4. Amendment No. 193 to NPF-29
5. Amendment No. 271 to NPF-26
6. Amendment No. 248 to NPF-64
7. Amendment No. 249 to DPR-20
8. Amendment No. 239 to DPR-35
9. Amendment No. 178 to DPR-47
10. Amendment No. 253 to DPR-28
11. Amendment No. 240 to DPR-38
12. Safety Evaluation

cc w/encls: Distribution via Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ENTERGY OPERATIONS, INC.

DOCKET NO. 50-313

ARKANSAS NUCLEAR ONE, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 248
Renewed License No. DPR-51

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Operations, Inc. (the licensee), dated December 13, 2011, as supplemented by letters dated May 21 and November 20, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

Enclosure 1

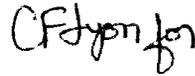
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-51 is hereby amended to read as follows:

- (2) Technical Specifications

- The Technical Specifications contained in Appendix A, as revised through Amendment No. 248, are hereby incorporated in the renewed license. EOI shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 120 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License No. DPR-51
and Technical Specifications

Date of Issuance: December 28, 2012

ATTACHMENT TO LICENSE AMENDMENT NO. 248

ARKANSAS NUCLEAR ONE, UNIT NO. 1

RENEWED FACILITY OPERATING LICENSE NO. DPR-51

DOCKET NO. 50-313

Replace the following pages of the Renewed Facility Operating License No. DPR-51 and Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Operating License

REMOVE

-3-

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-3-

Technical Specifications

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5.0-4

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5.0-4

- (5) EOI, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
 - (6) EOI, pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- c. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level
EOI is authorized to operate the facility at steady state reactor core power levels not in excess of 2568 megawatts thermal.
 - (2) Technical Specifications
The Technical Specifications contained in Appendix A, as revised through Amendment No. 248, are hereby incorporated in the renewed license. EOI shall operate the facility in accordance with the Technical Specifications.
 - (3) Safety Analysis Report
The licensee's SAR supplement submitted pursuant to 10 CFR 54.21(d), as revised on March 14, 2001, describes certain future inspection activities to be completed before the period of extended operation. The licensee shall complete these activities no later than May 20, 2014.
 - (4) Physical Protection
EOI shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans, including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contains Safeguards Information protected under 10 CFR 73.21, is entitled: "Arkansas Nuclear One Physical Security Plan, Training and Qualifications Plan, and Safeguards Contingency Plan," as submitted on May 4, 2006.

5.0 ADMINSTRATIVE CONTROLS

5.2 Organization

- c. Shift crew composition may be less than the minimum requirement of 10 CFR 50.54(m)(2)(i) for one unit, one control room, and 5.2.2.a and 5.2.2.f for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
 - d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the required position.
 - e. The operations manager or assistant operations manager shall hold an SRO license.
 - f. When in MODES 1, 2, 3, or 4 an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.
-

5.0 ADMINISTRATIVE CONTROLS

5.3 Unit Staff Qualifications

- 5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Energy Quality Assurance Program Manual (QAPM).
- 5.3.2 For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed Reactor Operator (RO) are those individuals who, in addition to meeting the requirements of Specification 5.3.1, perform the functions described in 10 CFR 50.54(m).
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UNITED STATES
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WASHINGTON, D.C. 20555-0001

ENTERGY OPERATIONS, INC.

DOCKET NO. 50-368

ARKANSAS NUCLEAR ONE, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 296
Renewed License No. NPF-6

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Operations, Inc. (the licensee), dated December 13, 2011, as supplemented by letters dated May 21 and November 20, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

Enclosure 2

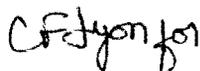
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-6 is hereby amended to read as follows:

- (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 296, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 120 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License No. NPF-6
Technical Specifications

Date of Issuance: December 28, 2012

ATTACHMENT TO LICENSE AMENDMENT NO. 296

ARKANSAS NUCLEAR ONE, UNIT NO. 2

RENEWED FACILITY OPERATING LICENSE NO. NPF-6

DOCKET NO. 50-368

Replace the following pages of the Renewed Facility Operating License No. NPF-6 and Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Operating License

REMOVE

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Technical Specifications

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- (4) EOI, pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (5) EOI, pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (6) EOI, pursuant to the Act and 10 CFR Parts 30 and 70 to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed license shall be deemed to contain and is subject to conditions specified in the following Commission regulations in 10 CFR Chapter I; Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

EOI is authorized to operate the facility at steady state reactor core power levels not in excess of 3026 megawatts thermal. Prior to attaining this power level EOI shall comply with the conditions in Paragraph 2.C.(3).

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 296, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

Exemptive 2nd paragraph of 2.C.2 deleted per Amendment 20, 3/3/81.

(3) Additional Conditions

The matters specified in the following conditions shall be completed to the satisfaction of the Commission within the stated time periods following issuance of the renewed license or within the operational restrictions indicated. The removal of these conditions shall be made by an amendment to the renewed license supported by a favorable evaluation by the Commission.

2.C.(3)(a) Deleted per Amendment 24, 6/19/81.

ADMINISTRATIVE CONTROLS

6.2.2 UNIT STAFF

- a. A non-licensed operator shall be on site when fuel is in the reactor and two additional non-licensed operators shall be on site when the reactor is in MODES 1, 2, 3, or 4.
- b. The minimum shift crew composition for licensed operators shall meet the minimum staffing requirements of 10 CFR 50.54(m)(2)(i) for one unit, one control room.
- c. Shift crew composition may be less than the minimum requirement of 10 CFR 50.54(m)(2)(i) for one unit, one control room, and 6.2.2.a and 6.2.2.f for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the required position.
- e. The operations manager or the assistant operations manager shall hold a SRO license.
- f. When in MODES 1, 2, 3, or 4 an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS

- 6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).
- 6.3.2 For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed Reactor Operator (RO) are those individuals who, in addition to meeting the requirements of Specification 6.3.1, perform the functions described in 10 CFR 50.54(m).

6.4 PROCEDURES

- 6.4.1 Written procedures shall be established, implemented, and maintained covering the following activities:
- a. The applicable procedures recommended in Regulatory Guide 1.33, Revision 2, Appendix A, February 1978;
 - b. The emergency operating procedures required to implement the requirements of NUREG-0737 and NUREG-0737, Supplement 1, as stated in Section 7.1 of Generic Letter 82-33;
 - c. Fire Protection Program implementation;
 - d. All programs specified in Specification 6.5; and
 - e. Modification of core protection calculator (CPC) addressable constants. These procedures shall include provisions to ensure that sufficient margin is maintained in CPC type I addressable constants to avoid excessive operator interaction with the CPCs during reactor operation.

Modifications to the CPC software (including changes of algorithms and fuel cycle specific data) shall be performed in accordance with the most recent version of "CPC Protection Algorithm Software Change Procedure," CEN-39(A)-P, which has been determined to be applicable to the facility. Additions or deletions to CPC addressable constants or changes to addressable constant software limit values shall not be implemented without prior NRC approval.



UNITED STATES
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WASHINGTON, D.C. 20555-0001

ENERGY NUCLEAR FITZPATRICK, LLC

AND ENERGY NUCLEAR OPERATIONS, INC.

DOCKET NO. 50-333

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 304
Renewed Facility Operating License No. DPR-59

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Operations, Inc. (the licensee), dated December 13, 2011, as supplemented by letters dated May 21 and November 20, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

Enclosure 3

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-59 is hereby amended to read as follows:

- (2) Technical Specifications

- The Technical Specifications contained in Appendix A, as revised through Amendment No. 304, are hereby incorporated in the renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 120 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George A. Wilson, Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License and Technical
Specifications

Date of Issuance: December 28, 2012

ATTACHMENT TO LICENSE AMENDMENT NO. 304
JAMES A. FITZPATRICK NUCLEAR POWER PLANT
RENEWED FACILITY OPERATING LICENSE NO. DPR-59
DOCKET NO. 50-333

Replace the following page of the License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove Page

Insert Page

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-3-

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages

Insert Pages

5.2-2

5.2-2

5.3-1

5.3-1

- (4) ENO pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use, at any time, any byproduct, source and special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration; or associated with radioactive apparatus, components or tools..
 - (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional considerations specified or incorporated below:
- (1) Maximum Power Level
ENO is authorized to operate the facility at steady state reactor core power levels not in excess of 2536 megawatts (thermal).
 - (2) Technical Specifications
The Technical Specifications contained in Appendix A, as revised through Amendment No. 304, are hereby incorporated in the renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.
 - (3) Fire Protection
ENO shall implement and maintain in effect all provisions of the approved fire protections program as described in the Final Safety Analysis Report for the facility and as approved in the SER dated November 20, 1972; the SER Supplement No. 1 dated February 1, 1973; the SER Supplement No. 2 dated October 4, 1974; the SER dated August 1, 1979; the SER Supplement dated October 3, 1980; the SER Supplement dated February 13, 1981; the NRC Letter dated February 24, 1981; Technical Specification Amendments 34 (dated January 31, 1978), 80 (dated May 22, 1984), 134 (dated July 19, 1989), 135 (dated September 5, 1989), 142 (dated October 23, 1989), 164 (dated August 10, 1990), 176 (dated January 16, 1992), 177 (dated February 10, 1992), 186 (dated February 19, 1993), 190 (dated June 29, 1993), 191 (dated July 7, 1993), 206 (dated February 28, 1994), and 214 (dated June 27, 1994); and NRC Exemptions and associated safety evaluations dated April 26, 1983, July 1, 1983, January 11, 1985, April 30, 1986, September 15, 1986 and September 10, 1992 subject to the following provision:

Renewed License No. DPR-59
Amendment No. 304

5.2 Organization

5.2.2 Plant Staff (continued)

- b. Shift crew composition may be less than the minimum requirement of 10 CFR 50.54(m)(2)(i) and 5.2.2.a and 5.2.2.f for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
 - c. A radiation protection technician shall be on site when fuel is in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the required position.
 - d. Deleted
 - e. The operations manager or assistant operations manager shall hold an SRO license.
 - f. When in MODES 1, 2, or 3 an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.
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5.0 ADMINISTRATIVE CONTROLS

5.3 Plant Staff Qualifications

- 5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).
- 5.3.2 For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed Reactor Operator (RO) are those individuals who, in addition to meeting the requirements of TS 5.3.1, perform the functions described in 10 CFR 50.54(m).
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ENTERGY OPERATIONS, INC.

SYSTEM ENERGY RESOURCES, INC.

SOUTH MISSISSIPPI ELECTRIC POWER ASSOCIATION

ENTERGY MISSISSIPPI, INC.

DOCKET NO. 50-416

GRAND GULF NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 193
License No. NPF-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Operations, Inc. (the licensee), dated December 13, 2011, as supplemented by letters dated May 21 and November 20, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

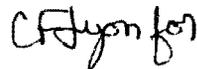
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-29, in part, is hereby amended to read as follows:

- (2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 193 are hereby incorporated into this license. Entergy Operations, Inc. shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 120 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Facility Operating
License No. NPF-29 and the
Technical Specifications

Date of Issuance: December 28, 2012

ATTACHMENT TO LICENSE AMENDMENT NO. 193

GRAND GULF NUCLEAR STATION, UNIT 1

FACILITY OPERATING LICENSE NO. NPF-29

DOCKET NO. 50-416

Replace the following pages of the Facility Operating License No. NPF-29 and the Appendix A, Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Facility Operating License

<u>Remove</u>	<u>Insert</u>
4	4

Technical Specifications

<u>Remove</u>	<u>Insert</u>
5.0-4	5.0-4
5.0-5	5.0-5

(b) SERI is required to notify the NRC in writing prior to any change in (i) the terms or conditions of any new or existing sale or lease agreements executed as part of the above authorized financial transactions, (ii) the GGNS Unit 1 operating agreement, (iii) the existing property insurance coverage for GGNS Unit 1 that would materially alter the representations and conditions set forth in the Staff's Safety Evaluation Report dated December 19, 1988 attached to Amendment No. 54. In addition, SERI is required to notify the NRC of any action by a lessor or other successor in interest to SERI that may have an effect on the operation of the facility.

C. The license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Entergy Operations, Inc. is authorized to operate the facility at reactor core power levels not in excess of 4408 megawatts thermal (100 percent power) in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 193 are hereby incorporated into this license. Entergy Operations, Inc. shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

During Cycle 19, GGNS will conduct monitoring of the Oscillation Power Range Monitor (OPRM). During this time, the OPRM Upscale function (Function 2.f of Technical Specification Table 3.3.1.1-1) will be disabled and operated in an "indicate only" mode and technical specification requirements will not apply to this function. During such time, Backup Stability Protection measures will be implemented via GGNS procedures to provide an alternate method to detect and suppress reactor core thermal hydraulic instability oscillations. Once monitoring has been successfully completed, the OPRM Upscale function will be enabled and technical specification requirements will be applied to the function; no further operating with this function in an "indicate only" mode will be conducted.

5.2 Organization

5.2.2 Unit Staff (continued)

- e. The operations manager or at least one operations middle manager shall hold an SRO license.
 - f. When in MODES 1, 2, or 3 an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.
-

5.0 ADMINISTRATIVE CONTROLS

5.3 Unit Staff Qualifications

5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).

5.3.2 For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed Reactor Operator (RO) are those individuals who, in addition to meeting the requirements of Specification 5.3.1, perform the functions described in 10 CFR 50.54(m).



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ENTERGY NUCLEAR INDIAN POINT 2, LLC

ENTERGY NUCLEAR OPERATIONS, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 271
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Nuclear Operations, Inc. (ENO, the licensee), dated December 13, 2011, as supplemented by letters dated May 21 and November 20, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

- (2) Technical Specifications

- The Technical Specifications contained in Appendices A, B, and C, as revised through Amendment No. 271, are hereby incorporated in the license. ENO shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 120 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "G. Wilson for," written in a cursive style.

George A. Wilson, Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the License and
Technical Specifications

Date of Issuance: December 28, 2012

ATTACHMENT TO LICENSE AMENDMENT NO. 271
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
FACILITY OPERATING LICENSE NO. DPR-26
DOCKET NO. 50-247

Replace the following page of the License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove Page

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-3-

-3-

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages

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5.2-2
5.3-1

5.2-2
5.3-1

instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

- (4) ENO pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; Amdt. 42
10-17-78
- (5) ENO pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility and Indian Point Nuclear Generating Unit No. 3 (IP3). Amdt. 220
09-06-01

C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

ENO is authorized to operate the facility at steady state reactor core power levels not in excess of 3216 megawatts thermal. Amdt. 241
10-27-2004

(2) Technical Specifications

The Technical Specifications contained in Appendices A, B, and C, as revised through Amendment No. 271, are hereby incorporated in the license. ENO shall operate the facility in accordance with the Technical Specifications.

(3) The following conditions relate to the amendment approving the conversion to Improved Standard Technical Specifications:

- 1. This amendment authorizes the relocation of certain Technical Specification requirements and detailed information to licensee controlled documents as described in Table R, "Relocated Technical Specifications from the CTS," and Table LA, "Removed Details and Less Restrictive administrative Changes to the CTS" attached to the NRC staff's Safety Evaluation enclosed with this amendment. The relocation of requirements and detailed information shall be completed on or before the implementation of this amendment.

5.2 Organization

5.2.2 Unit Staff (continued)

- b. Shift crew composition may be less than the minimum requirement of 10 CFR 50.54(m)(2)(i) and 5.2.2.a and 5.2.2.f for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
 - c. A radiation protection technician shall be on site when fuel is in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the required position.
 - d. Not Used
 - e. The operations manager or assistant operations manager shall hold an SRO license.
 - f. When in MODES 1, 2, 3, or 4 an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.
-

5.0 ADMINISTRATIVE CONTROLS

5.3 Unit Staff Qualifications

5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).

5.3.2 For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed reactor operator (RO) are those individuals who, in addition to meeting the requirements of TS 5.3.1, perform the functions described in 10 CFR 50.54(m).



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ENTERGY NUCLEAR INDIAN POINT 3, LLC

ENTERGY NUCLEAR OPERATIONS, INC.

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 248
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Nuclear Operations, Inc. (ENO, the licensee), dated December 13, 2011, as supplemented by letters dated May 21 and November 20, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

- (2) Technical Specifications

- The Technical Specifications contained in Appendices A, B, and C, as revised through Amendment No. 248, are hereby incorporated in the license. ENO shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 120 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George A. Wilson, Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the License and
Technical Specifications

Date of Issuance: December 28, 2012

ATTACHMENT TO LICENSE AMENDMENT NO. 248

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Replace the following page of the License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove Page

-3-

Insert Page

-3-

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

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5.0-5

- (4) ENO pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; Amdt. 203
11/27/00
- (5) ENO pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility. Amdt. 203
11/27/00
- C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
 - (1) Maximum Power Level

ENO is authorized to operate the facility at steady state reactor core power levels not in excess of 3216 megawatts thermal (100% of rated power).
 - (2) Technical Specifications

The Technical Specifications contained in Appendices A, B, and C, as revised through Amendment No. 248 are hereby incorporated in the License. ENO shall operate the facility in accordance with the Technical Specifications.
 - (3) (DELETED) Amdt. 205
2-27-01
 - (4) (DELETED) Amdt. 205
2-27-01
- D. (DELETED) Amdt.46
2-16-83
- E. (DELETED) Amdt.37
5-14-81
- F. This amended license is also subject to appropriate conditions by the New York State Department of Environmental Conservation in its letter of May 2, 1975, to Consolidated Edison Company of New York, Inc., granting a Section 401 certification under the Federal Water Pollution Control Act Amendments of 1972.

5.2 Organization

5.2.2 Unit Staff (continued)

- e. The operations manager or assistant operations manager shall hold an SRO license.
 - f. When in MODES 1, 2, 3, or 4 an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.
-

5.0 ADMINISTRATIVE CONTROLS

5.3 Unit Staff Qualifications

- 5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).
- 5.3.2 For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed Reactor Operator (RO) are those individuals who, in addition to meeting the requirements of TS 5.3.1, perform the functions described in 10 CFR 50.54(m).
-



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ENTERGY NUCLEAR OPERATIONS, INC.

DOCKET NO. 50-255

PALISADES NUCLEAR PLANT

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 249
License No. DPR-20

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Operations, Inc. (the licensee), dated December 13, 2011, as supplemented by letters dated May 21 and November 20, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to the license amendment and Paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-20, in part, is hereby amended to read as follows:
 - (2) The Technical Specifications contained in Appendix A, as revised through Amendment No. 249, and the Environmental Protection Plan contained in Appendix B are hereby incorporated in the license. ENO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.
3. The license amendment is effective as of its date of issuance and shall be implemented within 120 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert D. Carlson, Chief
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License and
Technical Specifications

Date of Issuance: December 28, 2012

ATTACHMENT TO LICENSE AMENDMENT NO. 249

PALISADES NUCLEAR PLANT

RENEWED FACILITY OPERATING LICENSE NO. DPR-20

DOCKET NO. 50-255

Replace the following page of the Renewed Facility Operating License No. DPR-20 with the attached revised page. The changed area is identified by a marginal line.

REMOVE

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Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

INSERT

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- (1) Pursuant to Section 104b of the Act, as amended, and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," (a) ENP to possess and use, and (b) ENO to possess, use and operate, the facility as a utilization facility at the designated location in Van Buren County, Michigan, in accordance with the procedures and limitation set forth in this license;
 - (2) ENO, pursuant to the Act and 10 CFR Parts 40 and 70, to receive, possess, and use source and special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Updated Final Safety Analysis Report, as supplemented and amended;
 - (3) ENO, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use byproduct, source, and special nuclear material as sealed sources for reactor startup, reactor instrumentation, radiation monitoring equipment calibration, and fission detectors in amounts as required;
 - (4) ENO, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material for sample analysis or instrument calibration, or associated with radioactive apparatus or components; and
 - (5) ENO, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operations of the facility.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations in 10 CFR Chapter I and is subject to all applicable provisions of the Act; to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) ENO is authorized to operate the facility at steady-state reactor core power levels not in excess of 2565.4 Megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.
 - (2) The Technical Specifications contained in Appendix A, as revised through Amendment No. 249, and the Environmental Protection Plan contained in Appendix B are hereby incorporated in the license. ENO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.
 - (3) ENO shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility and as approved in the SERs dated 09/01/78, 03/19/80, 02/10/81, 05/26/83, 07/12/85, 01/29/86, 12/03/87, and 05/19/89 and subject to the following provisions:

Renewed License No. DPR-20
Amendment No. 249

5.2 Organization

Plant Staff (continued)

- c. Shift crew composition may be less than the minimum requirement of 10 CFR 50.54(m)(2)(i), and 5.2.2.a and 5.2.2.g for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the requirements.
 - d. A radiation safety technician shall be on site when fuel is in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the required position.
 - e. Not Used
 - f. The operations manager or an assistant operations manager shall hold an SRO license. The individual holding the SRO license shall be responsible for directing the activities of the licensed operators.
 - g. When in MODES 1, 2, 3, or 4 an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.
-

5.0 ADMINISTRATIVE CONTROLS

5.3 Plant Staff Qualifications

- 5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Energy Quality Assurance Program Manual (QAPM).
- 5.3.2 (Deleted)
- 5.3.3 (Deleted)
- 5.3.4 (Deleted)
- 5.3.5 For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed reactor operator (RO) are those individuals who, in addition to meeting the requirements of TS 5.3.1, perform the functions described in 10 CFR 50.54(m).
-



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ENTERGY NUCLEAR GENERATION COMPANY

ENTERGY NUCLEAR OPERATIONS, INC.

DOCKET NO. 50-293

PILGRIM NUCLEAR POWER STATION

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 239
License No. DPR-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Operations, Inc. (the licensee), dated December 13, 2011, as supplemented by letters dated May 21 and November 20, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

Enclosure 8

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Renewed Facility Operating License No. DPR-35 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 239, are hereby incorporated in the renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 120 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "G. Wilson for," with a stylized, cursive script.

George A. Wilson, Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the License and
Technical Specifications

Date of Issuance: December 28, 2012

ATTACHMENT TO LICENSE AMENDMENT NO. 239

PILGRIM NUCLEAR POWER STATION

RENEWED FACILITY OPERATING LICENSE NO. DPR-35

DOCKET NO. 50-293

Replace the following page of the Renewed Facility Operating License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove

Insert

-3-

-3-

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

Insert

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5.0-4

provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

A. Maximum Power Level

ENO is authorized to operate the facility at steady state power levels not to exceed 2028 megawatts thermal.

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 239, are hereby incorporated in the renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

C. Records

ENO shall keep facility operating records in accordance with the requirements of the Technical Specifications.

D. Equalizer Valve Restriction – DELETED

E. Recirculation Loop Inoperable – DELETED

F. Fire Protection

ENO shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility and as approved in the SER dated December 21, 1978 as supplemented subject to the following provision:

ENO may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

G. Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contain Safeguards Information protected under 10 CFR 73.21, is entitled: "Pilgrim Nuclear Power Station Physical Security, Training and Qualification, and Safeguards Contingency Plan, Revision 0" submitted by letter dated October 13, 2004, as supplemented by letter dated May 15, 2006.

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved cyber security plan (CSP), including changes made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The licensee's CSP was approved by License Amendment No. 236, as supplemented by a change approved by Amendment No. 238.

5.2 Organization

5.2.2 Unit Staff (continued)

- b. At least one licensed Reactor Operator (RO) shall be present in the control room when fuel is in the reactor. In addition, while the unit is in an operational mode other than Cold Shutdown or Refueling, at least one licensed Senior Reactor Operator (SRO) shall be present in the control room.
 - c. At least two licensed ROs shall be present in the control room during reactor startup, scheduled reactor shutdown and during recovery from reactor trips.
 - d. Shift crew composition may be less than the minimum requirement of 10 CFR 50.54(m)(2)(i) and 5.2.2.a and 5.2.2.i for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
 - e. Higher grade licensed operators may take the place of lower grade licensed or unlicensed personnel.
 - f. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the required position.
 - g. Deleted
 - h. The operations manager or assistant operations manager shall hold a Senior Reactor Operator License.
 - i. When the unit is in an operational mode other than Cold Shutdown or Refueling, an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.
-

5.0 ADMINISTRATIVE CONTROLS

5.3 Unit Staff Qualifications

- 5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).
- 5.3.2 For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed Reactor Operator (RO) are those individuals who, in addition to meeting the requirements of Specification 5.3.1, perform the functions described in 10 CFR 50.54(m).
-



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ENERGY GULF STATES LOUISIANA, LLC

AND

ENERGY OPERATIONS, INC.

DOCKET NO. 50-458

RIVER BEND STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 178
License No. NPF-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Operations, Inc. (the licensee), dated December 13, 2011, as supplemented by letters dated May 21 and November 20, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

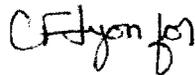
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-47 is hereby amended to read as follows:

- (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 178 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. EOI shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 120 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Facility Operating
License No. NPF-47 and
Technical Specifications

Date of Issuance: December 28, 2012

ATTACHMENT TO LICENSE AMENDMENT NO. 178

RIVER BEND STATION, UNIT 1

FACILITY OPERATING LICENSE NO. NPF-47

DOCKET NO. 50-458

Replace the following pages of the Facility Operating License No. NPF-47 and Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by Amendment number and contain marginal lines indicating the areas of change.

Facility Operating License

<u>Remove</u>	<u>Insert</u>
-3-	-3-

Technical Specifications

<u>Remove</u>	<u>Insert</u>
5.0-4	5.0-4
5.0-5	5.0-5

- (3) EOI, pursuant to the Act and 10 CFR Part 70, to receive, possess and to use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
- (4) EOI, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (5) EOI, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (6) EOI, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

EOI is authorized to operate the facility at reactor core power levels not in excess of 3091 megawatts thermal (100% rated power) in accordance with the conditions specified herein. The items identified in Attachment 1 to this license shall be completed as specified. Attachment 1 is hereby incorporated into this license.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 178 and the Environmental Protection Plan contained in Appendix 8, are hereby incorporated in the license. EOI shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

5.2 Organization

5.2.2 Unit Staff (continued)

- f. The operations manager or at least one operations middle manager shall hold an SRO license.
 - g. When in MODES 1, 2, or 3 an individual shall provide advisory technical support to the unit operations crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.
-
-

5.0 ADMINISTRATIVE CONTROLS

5.3 Unit Staff Qualifications

5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).

5.3.2 For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed Reactor Operator (RO) are those individuals who, in addition to meeting the requirements of Specification 5.3.1, perform the functions described in 10 CFR 50.54(m).



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ENTERGY NUCLEAR VERMONT YANKEE, LLC

AND ENTERGY NUCLEAR OPERATIONS, INC.

DOCKET NO. 50-271

VERMONT YANKEE NUCLEAR POWER STATION

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 253
License No. DPR-28

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by Entergy Nuclear Vermont Yankee, LLC and Entergy Nuclear Operations, Inc. (the licensee) dated December 13, 2011, as supplemented by letters dated May 21 and November 20, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Renewed Facility Operating License No. DPR-28 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 253, are hereby incorporated in the license. Entergy Nuclear Operations, Inc. shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 120 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "G. Wilson for," with a horizontal line extending from the end of the signature.

George A. Wilson, Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the License and
Technical Specifications

Date of Issuance: December 28, 2012

ATTACHMENT TO LICENSE AMENDMENT NO. 253
VERMONT YANKEE NUCLEAR POWER STATION
RENEWED FACILITY OPERATING LICENSE NO. DPR-28
DOCKET NO. 50-271

Replace the following page of the Renewed Facility Operating License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove

Insert

-3-

-3-

Replace the following page(s) of the Appendix A Technical Specifications with the attached revised page(s). The revised page(s) are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

Insert

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- D. Entergy Nuclear Operations, Inc., pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any Byproduct, source, or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components.
 - E. Entergy Nuclear Operations, Inc., pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not to separate, such byproduct and special nuclear material as may be produced by operation of the facility.
3. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Section 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

A. Maximum Power Level

Entergy Nuclear Operations, Inc. is authorized to operate the facility at reactor core power levels not to exceed 1912 megawatts thermal in accordance with the Technical Specifications (Appendix A) appended hereto.

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 253 are hereby incorporated in the license. Entergy Nuclear Operations, Inc. shall operate the facility in accordance with the Technical Specifications.

C. Reports

Entergy Nuclear Operations, Inc. shall make reports in accordance with the requirements of the Technical Specifications.

D. This paragraph deleted by Amendment No. 226.

E. Environmental Conditions

Pursuant to the Initial Decision of the presiding Atomic Safety and Licensing Board issued February 27, 1973, the following conditions for the protection of the environment are incorporated herein:

1. This paragraph deleted by Amendment No. 206, October 22, 2001.
2. This paragraph deleted by Amendment 131, 10/07/91.

VYNPS

6.2 ORGANIZATION (Cont'd)

4. The individuals who train the operating staff, carry out health physics, or perform quality assurance functions may report to the appropriate on-site manager; however, these individuals shall have sufficient organizational freedom to ensure their independence from operating pressures.

B. Unit Staff

The unit staff organization shall include the following:

1. A non-licensed operator shall be assigned when the reactor contains fuel and an additional non-licensed operator shall be assigned during Plant Startup and Normal Operation.
2. At least one licensed Reactor Operator (RO) or one licensed Senior Reactor Operator (SRO) shall be present in the control room when fuel is in the reactor.
3. When the unit is in Plant Startup or Normal Operation, at least one licensed Senior Reactor Operator (SRO) and one licensed Reactor Operator (RO), or two licensed Senior Reactor Operators, shall be present in the control room.
4. Shift crew composition shall meet the requirements stipulated herein and in 10 CFR 50.54(m). Shift crew composition may be less than the minimum requirement of 10 CFR 50.54(m)(2)(i) and Specifications 6.2.B.1 and 6.2.B.8 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members, provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
5. An individual qualified in radiation protection procedures shall be present on-site when there is fuel in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the required position.
6. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety related functions (e.g., licensed SROs, licensed ROs, radiation protection technicians, auxiliary operators, and key maintenance personnel).
7. The operations manager or an assistant operations manager shall hold an SRO license.
8. When the unit is in Plant Startup or Normal Operation an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.

6.2 ORGANIZATION (Cont'd)

C. Unit Staff Qualifications

1. Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).
2. For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed Reactor Operator (RO) are those individuals who, in addition to meeting the requirements of Specification 6.2.C.1, perform the functions described in 10 CFR 50.54(m).

6.3 ACTION TO BE TAKEN IF A SAFETY LIMIT IS EXCEEDED

Applies to administrative action to be followed in the event a safety limit is exceeded.

If a safety limit is exceeded, the reactor shall be shutdown immediately.

6.4 PROCEDURES

Written procedures shall be established, implemented, and maintained covering the following activities:

- A. Normal startup, operation and shutdown of systems and components of the facility.
- B. Refueling operations.
- C. Actions to be taken to correct specific and foreseen potential malfunctions of systems or components, suspected Primary System leaks and abnormal reactivity changes.
- D. Emergency conditions involving potential or actual release of radioactivity.
- E. Preventive and corrective maintenance operations which could have an effect on the safety of the reactor.
- F. Surveillance and testing requirements.
- G. Fire protection program implementation.
- H. Process Control Program in-plant implementation.
- I. Off-Site Dose Calculation Manual implementation.

6.5 HIGH RADIATION AREA

As provided in paragraph 20.1601(c) of 10 CFR 20, the following controls shall be applied to high radiation areas in place of the controls required by paragraphs 20.1601(a) and 20.1601(b) of 10 CFR 20:

- A. High Radiation Areas with dose rates greater than 0.1 rem/hour at 30 centimeters, but not exceeding 1.0 rem/hour at 30 centimeters from the radiation source or from any surface penetrated by the radiation:



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ENTERGY OPERATIONS, INC.

DOCKET NO. 50-382

WATERFORD STEAM ELECTRIC STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 240
License No. NPF-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Operations, Inc. (the licensee), dated December 13, 2011, as supplemented by letters dated May 21 and November 20, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

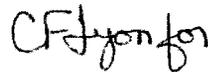
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.2 of Facility Operating License No. NPF-38 is hereby amended to read as follows:

2. Technical Specifications and Environmental Protection Plan

- The Technical Specifications contained in Appendix A, as revised through Amendment No. 240, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. EOI shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 120 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Facility Operating
License No. NPF-38 and
Technical Specifications

Date of Issuance: December 28, 2012

ATTACHMENT TO LICENSE AMENDMENT NO. 240

WATERFORD STEAM ELECTRIC STATION, UNIT 3

TO FACILITY OPERATING LICENSE NO. NPF-38

DOCKET NO. 50-382

Replace the following pages of the Facility Operating License and Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Facility Operating License

<u>REMOVE</u>	<u>INSERT</u>
-4-	-4-

Technical Specifications

<u>REMOVE</u>	<u>INSERT</u>
6-6	6-6
6-6a	----
6-7	6-7

or indirectly any control over (i) the facility, (ii) power or energy produced by the facility, or (iii) the licensees of the facility. Further, any rights acquired under this authorization may be exercised only in compliance with and subject to the requirements and restrictions of this operating license, the Atomic Energy Act of 1954, as amended, and the NRC's regulations. For purposes of this condition, the limitations of 10 CFR 50.81, as now in effect and as they may be subsequently amended, are fully applicable to the equity investors and any successors in interest to the equity investors, as long as the license for the facility remains in effect.

- (b) Entergy Louisiana, LLC (or its designee) to notify the NRC in writing prior to any change in (i) the terms or conditions of any lease agreements executed as part of the above authorized financial transactions, (ii) any facility operating agreement involving a licensee that is in effect now or will be in effect in the future, or (iii) the existing property insurance coverages for the facility, that would materially alter the representations and conditions, set forth in the staff's Safety Evaluation enclosed to the NRC letter dated September 18, 1989. In addition, Entergy Louisiana, LLC or its designee is required to notify the NRC of any action by equity investors or successors in interest to Entergy Louisiana, LLC that may have an effect on the operation of the facility.

- C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

- 1. Maximum Power Level

EOI is authorized to operate the facility at reactor core power levels not in excess of 3716 megawatts thermal (100% power) in accordance with the conditions specified herein.

- 2. Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 240, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. EOI shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

ADMINISTRATIVE CONTROLS

6.2.3 Not Used

6.2.4 SHIFT TECHNICAL ADVISOR

When in MODES 1, 2, 3, or 4 an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS

- 6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).
- 6.3.2 For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed Reactor Operator (RO) are those individuals who, in addition to meeting the requirements of Specification 6.3.1, perform the functions described in 10 CFR 50.54(m).

6.4 NOT USED.

6.5 PROGRAMS

The following programs shall be established, implemented, and maintained.

6.5.1 through 6.5.4 will be used later.

6.5.5 COMPONENT CYCLIC OR TRANSIENT LIMIT

This program provides controls to track Technical Requirements Manual Section 5.7 cyclic and transient occurrences to ensure that components are maintained within the design limits.

6.5.6 Will be used later.

6.5.7 REACTOR COOLANT PUMP FLYWHEEL INSPECTION PROGRAM

This program shall provide for the inspection of each reactor coolant pump flywheel per the recommendation of Regulatory Position C.4.b of Regulatory Guide 1.14, Revision 1, August 1975. The volumetric examination per Regulatory Position C.4.b.1 will be performed on approximately 10-year intervals.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 248 AND 296 TO RENEWED
FACILITY OPERATING LICENSE NOS. DPR-51 AND NPF-6 FOR
ARKANSAS NUCLEAR ONE, UNIT NOS. 1 AND 2;
AMENDMENT NO. 304 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-59
FOR JAMES A. FITZPATRICK NUCLEAR POWER PLANT;
AMENDMENT NO. 193 TO FACILITY OPERATING LICENSE NO. NPF-29 FOR
GRAND GULF NUCLEAR STATION, UNIT 1;
AMENDMENT NOS. 271 AND 248 TO FACILITY OPERATING LICENSE NOS. DPR-26
AND DPR-64 FOR INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2 AND 3;
AMENDMENT NO. 249 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-20
FOR PALISADES NUCLEAR PLANT;
AMENDMENT NO. 239 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-35
FOR PILGRIM NUCLEAR POWER STATION;
AMENDMENT NO. 178 TO FACILITY OPERATING LICENSE NO. NPF-47 FOR
RIVER BEND STATION, UNIT 1;
AMENDMENT NO. 253 TO RENEWED FACILITY OPERATING LICENSE DPR-28 FOR
VERMONT YANKEE NUCLEAR POWER STATION; AND
AMENDMENT NO. 240 TO FACILITY OPERATING LICENSE NO. NPF-38 FOR
WATERFORD STEAM ELECTRIC STATION, UNIT 3
BIG ROCK POINT PLANT
INDIAN POINT NUCLEAR GENERATING UNIT NO. 1
DOCKET NOS. 50-313, 72-13, 50-368, 50-155, 72-43, 50-333, 72-12, 50-416,
72-50, 50-003, 50-247, 72-51, 50-286, 50-255, 72-7, 50-293, 50-458, 72-49,
50-271, 72-59, 50-382 AND 72-75

1.0 INTRODUCTION

By application dated December 13, 2011 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML11356A278), as supplemented by letters dated May 21 and November 20, 2012 (ADAMS Accession Nos. ML12145A195 and ML12328A039, respectively), Entergy Operations, Inc. (Entergy, the licensee), submitted a fleet license amendment request. The request consisted of changes to the Technical Specifications (TSs) for Arkansas Nuclear

One, Units 1 and 2, Grand Gulf Nuclear Station, Unit 1, River Bend Station, Unit 1, Waterford Steam Electric Station, Unit 3, James A. FitzPatrick Nuclear Power Plant, Pilgrim Nuclear Power Station, Indian Point Nuclear Generating Unit Nos. 1, 2, and 3, Palisades Nuclear Plant, and Vermont Yankee Nuclear Power Station.

The licensee proposed changes to the Quality Assurance Program Manual (QAPM) and Technical Specifications (TS) to standardize unit staff qualification requirements for the Entergy fleet. Some proposed changes to the QAPM are a reduction in commitment and, in accordance with paragraph 50.54(a)(4) of Title 10 of the *Code of Federal Regulations* (10 CFR), U.S. Nuclear Regulatory Commission (NRC) approval is required prior to implementation. The related TS changes for unit staff qualifications are requested in accordance with 10 CFR 50.90.

The supplemental letters dated May 21 and November 20, 2012, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the NRC staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on March 20, 2012 (77 FR 16274).

Indian Point Nuclear Generating Unit No. 1 (IP1) is currently defueled and operates as a support facility for Indian Point Nuclear Generating Unit No. 2 (IP2). IP1 and IP2 are physically contiguous and share a number of systems and facilities as well as a common operating organization. The IP1 TSs were amended on August 11, 2003, via Amendment No. 52 (ADAMS Accession No. ML032240294), to acknowledge this by incorporating applicable sections of the IP2 TS by reference. Regarding this proposed change, IP1 TS Section 3.2, "Organization," states, "The organization requirements are as specified in Appendix A to the Indian Point Nuclear Generating Unit No. 2 Facility Operating License No. DPR-26." Therefore, there are no specific changes to the IP1 TS pages.

Big Rock Point (BRP) is defueled, decommissioned, and, as specified in Condition C.(1) of the BRP operating license, the reactor is not licensed for power operation. As such, the BRP TS were amended to reflect a non-operating, decommissioned facility.

The qualifications for the operating staff at IP1 and BRP are specified in each facility's TS, which covers operation of each facility's independent spent fuel storage installation (ISFSI). Therefore, the changes to the Entergy QAPM and each facility's TS adequately address activities governed under 10 CFR Parts 50 and 72; no other changes to documents controlled by the NRC are necessary.

2.0 REGULATORY EVALUATION

The Commission's regulatory requirements related to the content of the TS are set forth in Section 50.36, "Technical specifications," of Title 10 of the *Code of Federal Regulations* (10 CFR). Pursuant to 10 CFR 50.36, TS are required to include items in the following five specific categories related to station operation: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls.

The Commission's regulatory requirements related to quality assurance (QA) programs are set forth in Appendix B, "Quality Assurance Criteria for Nuclear Power Plants and Fuel Reprocessing Plants," to 10 CFR Part 50, 10 CFR 50.34(b)(6)(ii), and 10 CFR 50.54(a).

Appendix B establishes QA requirements for the design, construction, and operation of structures, systems, and components (SSCs) of a facility. The pertinent requirements of Appendix B to 10 CFR Part 50 apply to all activities affecting the safety-related functions of those SSCs and include designing, purchasing, fabricating, handling, shipping, storing, cleaning, erecting, installing, inspecting, testing, operating, maintaining, repairing, refueling, and modifying.

The regulations in 10 CFR 50.34, "Content of applications; technical information," require that every applicant for an operating license include information in its Final Safety Analysis Report (FSAR) on the managerial and administrative controls to be used to ensure safe operation. The information on the controls shall also include a discussion of how the applicable requirements of Appendix B will be satisfied.

The regulations in 10 CFR 50.54 require each power plant subject to 10 CFR Part 50 Appendix B to implement a QA program and 10 CFR 50.54(a)(3) allows licensees to make changes to their QA program so long as those changes do not reduce commitments and the licensee submits a copy of the changes to the NRC. The current Entergy QAPM specifies the use of American National Standards Institute/American Nuclear Society (ANSI/ANS) 3.1-1978 standards for unit staff qualifications. However, the TS for the James A. FitzPatrick NPP, the Palisades NP, Pilgrim NPS, Waterford SES, Unit 3, and Vermont Yankee NPS specify the standards of ANSI/ANS N18.1-1971 for unit staff qualifications. The licensee proposed TS changes to be consistent with the QAPM. In addition, Entergy proposed to relocate exceptions to the standard from the TS to the QAPM. For example, the Radiation Protection Manager (RPM) qualifications, which are an exception to the standard, will be relocated to the QAPM. Changes to the exceptions following relocation would be limited by 10 CFR 50.54 to those that do not reduce commitments or that receive NRC approval prior to implementation.

The regulations in 10 CFR 50.120, "Training and qualification of nuclear power plant personnel," require licensees to establish, implement, and maintain a training program that meets specified requirements.

The regulations in 10 CFR Part 55, "Operator's Licenses," establish procedures and criteria for the issuance of licenses to operators and senior operators, provide for the terms and conditions upon which the Commission will issue or modify these licenses, and provide for the terms and conditions to maintain and renew these licenses. Section 55.4, "Definitions," provides a definition which states, in part, that

Actively performing the functions of an operator or senior operator means that an individual has a position on the shift crew that requires the individual to be licensed as defined in the facility's technical specifications, and that the individual carries out and is responsible for the duties covered by that position.

NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants: LWR Edition", Chapter 13, "Conduct of Operation," includes the following specific sections considered in this review:

- Section 13.1.1, "Management and Technical Support Organization," Revision 5 (ADAMS Accession No. ML070460302);
- Section 13.1.2-13.1.3, "Operating Organization," Revision 6 (ADAMS Accession No. ML070250009); and
- Section 13.2.1, "Reactor Operator Requalification Program; Reactor Operator Training," Revision 3 (ADAMS Accession No. ML070100636).

NUREG-0800, Section 17.5, "Quality Assurance Program Description - Design Certification, Early Site Permit and New License Applicants," Revision 0 (ADAMS Accession No. ML060180622), includes a discussion about the requirements for the qualification of QA personnel.

NRC Regulatory Guide 1.8, "Qualification and Training of Personnel for Nuclear Power Plants," Revision 2, April 1987 (ADAMS Accession No. ML003739928), and Revision 3, May 2000 (ADAMS Accession No. ML003706932), provides guidance regarding qualifications and training for nuclear power plant personnel.

3.0 TECHNICAL EVALUATION

3.1 Proposed TS Changes

3.1.1 Arkansas Nuclear One, Unit 1 (ANO-1)

Current TS 5.2.f states:

In MODES 1, 2, 3, or 4, an individual shall provide advisory technical support for the operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. This individual shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift.

Revised TS 5.2.f would state:

When in MODES 1, 2, 3, or 4 an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANS/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.

Current TS 5.3.1 states:

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI ANS 3.1- 1978 for comparable positions, except for the designated radiation protection manager, who shall meet or exceed the minimum qualifications of Regulatory Guide 1.8, September 1975.

Revised TS 5.3.1 would state:

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).

New TS 5.3.2 would be added:

For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed Reactor Operator (RO) are those individuals who, in addition to meeting the requirements of Specification 5.3.1, perform the functions described in 10 CFR 50.54(m).

3.1.2 Arkansas Nuclear One, Unit 2 (ANO-2)

Current TS 6.2.2.f states:

In MODES 1, 2, 3, or 4, an individual shall provide advisory technical support for the operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. This individual shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift.

Revised TS 5.2.f would state:

When in MODES 1, 2, 3, or 4 an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.

Current TS 6.3.1 states:

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI ANS 3.1-1978 for comparable positions, except for the designated radiation protection manager, who shall meet or exceed the minimum qualifications of Regulatory Guide 1.8, September 1975.

Revised TS 6.3.1 would state:

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).

New TS 6.3.2 would state:

For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed Reactor Operator (RO) are those individuals who, in addition meeting the requirements of Specification 6.3.1, perform the functions described in 10 CFR 50.54(m).

3.1.3 James A. FitzPatrick Nuclear Power Plant (FitzPatrick)

Current TS 5.2.2.f states:

When the plant is in MODE 1, 2, or 3, an individual shall provide advisory technical support to the shift supervisor (SS) in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the plant. This individual shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift, published in the October 28, 1985 Federal Register (50 FR 43621).

Revised TS 5.2.2.f would state:

When in MODES 1, 2, or 3 an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.

Current TS 5.3.1 states:

Each member of the plant staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions except for the radiation protection manager, who shall meet or exceed the qualifications of Regulatory Guide 1.8, Revision 1, September 1975.

Revised TS 5.3.1 would state:

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).

3.1.4 Grand Gulf Nuclear Station, Unit 1 (GGNS)

Current TS 5.2.2.f states:

The Shift Technical Advisor (STA) shall provide advisory technical support to the shift superintendent in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. In addition, the STA shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift.

Revised TS 5.2.2.f would state:

When in MODES 1, 2, or 3 an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.

Current TS 5.3.1 states:

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS-3.1-1978, except as clarified in the Quality Assurance Program Manual, for comparable positions as modified by Specification 5.2.2.f, except for the radiation protection manager and the STA, who shall meet or exceed the education and experience requirements of ANSI/ANS 3.1-1981 as endorsed by Regulatory Guide 1.8, Revision 2, 1987.

Revised TS 5.3.1 would state:

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).

New TS 5.3.2 would state:

For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed Reactor Operator (RO) are those individuals who, in addition to meeting the requirements of Specification 5.3.1, perform the functions described in 10 CFR 50.54(m).

3.1.5 Indian Point Nuclear Generating Unit 2 (IP2)

Current TS 5.2.2.f states:

An individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. This individual shall meet the

qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift. This position must be manned only when in MODES 1, 2, 3, or 4.

Revised TS 5.2.2.f would state:

When in MODES 1, 2, 3, or 4 an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.

Current TS 5.3.1 states:

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions, except for the following:

- a. The radiation protection manager shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975; and
- b. The operations manager shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 except for the SRO license requirement which shall be in accordance with Technical Specification 5.2.2.e.

Revised TS 5.3.1 would state:

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).

As indicated above, any change to the organizational requirements in the IP2 TS changes the IP1 TS organizational requirements.

3.1.6 Indian Point Nuclear Generating Unit 3 (IP3)

Current TS 5.2.2.f states:

An individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. This individual shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift. This position must be manned when in MODES 1, 2, 3, or 4 only.

Revised TS 5.2.2.f would state:

When in MODES 1, 2, 3, or 4 an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.

Current TS 5.3.1 states:

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions, except for the following:

- a. The radiation protection manager shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975; and
- b. The operations manager shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 except for the SRO license requirement which shall be in accordance with Technical Specification 5.2.2.e.

Revised TS 5.3.1 would state:

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).

3.1.7 Palisades Nuclear Plant

Current TS 5.2.2.g states:

An individual shall provide advisory technical support to the plant operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the plant. This individual shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift (Published in Federal Register 50 FR 43621, October 28, 1985).

Revised TS 5.2.2.g would state:

When in MODES 1, 2, 3, or 4 an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.

Current TS 5.3.1 states:

Each member of the plant staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions except for the education and experience eligibility requirements for operator license applicants, and changes thereto, shall be those previously reviewed and approved by the NRC, specifically those referenced in NRC Safety Evaluation dated October 24, 2003.

Revised TS 5.3.1 would state:

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).

Current TS 5.3.2 states:

The radiation safety manager shall meet the qualifications of a Radiation Protection Manager as defined in Regulatory Guide 1.8, September 1975. For the purpose of this section, "Equivalent," as utilized in Regulatory Guide 1.8 for the bachelor's degree requirement, may be met with four years of any one or combination of the following: (a) Formal schooling in science or engineering, or (b) operational or technical experience and training in nuclear power.

Revised TS 5.3.2 would state:

(Deleted)

Current TS 5.3.3 states:

The individual, required by Specification 5.2.2g, assigned to provide advisory technical support to the plant operations shift crew, shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift (Published in Federal Register 50 FR 43621, October 28, 1985).

Revised TS 5.3.3 would state:

(Deleted)

3.1.8 Pilgrim Nuclear Power Station

Current TS 5.2.2.i states:

An individual shall provide advisory technical support to the unit operations shift crew in the areas of engineering and accident assessment. This individual shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift. This individual with a Senior Reactor Operator license may simultaneously serve a required SRO position.

Revised TS 5.2.2.i would state:

When the unit is in an operational mode other than Cold Shutdown or Refueling, an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.

Current TS 5.3.1 states:

Each member of the unit staff shall meet or exceed the minimum qualifications as described in the American National Standards Institute N18.1-1971, "Selection and Training of Personnel for Nuclear Power Plants." In addition, the individual performing the function of Radiation Protection Manager shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

Revised TS 5.3.1 would state:

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).

New TS 5.3.2 would state:

For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed Reactor Operator (RO) are those individuals who, in addition to meeting the requirements of Specification 5.3.1, perform the functions described in 10 CFR 50.54(m).

3.1.9 River Bend Station (RBS)

Current TS 5.2.2.g states:

The Shift Technical Advisor (STA) shall provide advisory technical support to the shift superintendent in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. In addition, the STA shall meet the qualifications specified by the Commission Policy Statement on Engineering Expertise on Shift.

Revised TS 5.2.2.g would state:

When in MODES 1, 2, or 3 an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit.

This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.

Current TS 5.3.1 states:

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions, except for the radiation protection manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

Revised TS 5.3.1 would state:

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).

New TS 5.3.2 would state:

For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed Reactor Operator (RO) are those individuals who, in addition to meeting the requirements of Specification 5.3.1, perform the functions described in 10 CFR 50.54(m).

3.1.10 Vermont Yankee Nuclear Power Station (VY)

Current TS.6.2.B.8 states:

While the unit is in Plant Startup or Normal Operation, a shift engineer shall provide advisory technical support to the shift supervisor.

Revised TS.6.2.B.8 would state:

When the unit is in Plant Startup or Normal Operation an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.

Current TS 6.2.C states:

Each member of the unit staff shall meet or exceed the minimum qualifications of the American National Standards Institute N-18.1-1971, "Selection and Training of Personnel for Nuclear Power Plants," except for the radiation protection manager who shall meet the qualifications of Regulatory Guide 1.8, Revision 1 (September 1975) and the shift engineer, who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

- b. Option 2 - Dedicated STA Position. This option is satisfied by placing on each shift a dedicated Shift Technical Advisor (STA) who meets the STA criteria of NUREG-0737, Item I.A.1.1.

Revised TS 6.2.4 would state:

When in MODES 1, 2, 3, or 4 an individual shall provide advisory technical support to the unit operations shift crew in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operations of the unit. This individual shall meet the qualifications specified by ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Rev. 3, 2000.

Current TS 6.3.1 states:

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 except that:

- a. The Radiation Protection Superintendent shall meet or exceed the minimum qualifications of Regulatory Guide 1.8, September 1975.
- b. Personnel in the Health Physics, Chemistry and Radwaste Departments shall meet or exceed the minimum qualifications of ANSI N18.1-1971.
- c. The licensed Operators and Senior Operators shall also meet or exceed the minimum qualifications of 10 CFR Part 55.
- d. Personnel in the Nuclear Quality Assurance Department, and other staff personnel who perform inspection, examination, and testing functions, shall meet or exceed the minimum qualifications of Regulatory Guide 1.58, Rev. 1, September 1980. (Endorses ANSI N45.2.6-1978).

Revised TS 6.3.1 would state:

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Entergy Quality Assurance Program Manual (QAPM).

New TS 6.3.2 would state:

For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed Reactor Operator (RO) are those individuals who, in addition to meeting the requirements of Specification 6.3.1, perform the functions described in 10 CFR 50.54(m).

Current TS 6.4 states:

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Training Manager-Nuclear and shall meet or exceed the requirements and recommendations of Section 5.2 of ANSI 3.1-1978 and 10 CFR Part 55.

Revised TS 6.4 would state:

6.4 Not Used

3.2 NRC Staff Evaluation

3.2.1 Background

The Entergy QAPM provides an overview of the quality program controls which govern the operation and maintenance of Entergy's quality-related items and activities. The QAPM ensures conformance to Appendix B to 10 CFR Part 50, and applies to all of the units in Entergy's nuclear fleet. In accordance with 10 CFR 50.54(a)(4), Entergy requested NRC approval of proposed changes to unit staff qualification requirements contained in the QAPM. Some changes to the qualification requirements for the RPM and QA Manager are considered a reduction in commitments. Therefore, NRC approval is required prior to implementation. Other individual changes are clarifications or administrative changes that do not reduce commitments, but are included in the evaluation for completeness.

The Entergy QAPM and plant TSs specify requirements for unit staff qualifications. Exceptions to the requirements are also included in both the QAPM and the TS. Currently, Entergy's QAPM references ANSI/ANS 3.1-1978 as the overall standard for unit staff qualifications. ANSI/ANS 3.1-1978 was not officially endorsed by a regulatory guide; however, this standard was found acceptable as referenced in Entergy's QAPM and TSs for Entergy plants in accordance with an NRC staff safety evaluation dated November 6, 1998 (ADAMS Legacy Accession Nos. 9811170123 and 9811170129).

Entergy's use of exceptions in both the QAPM and the TSs has resulted in confusion over the applicability and extent to which requirements apply. Entergy proposed to revise the QAPM and each plant's TSs for consistency, and to allow the use of the QAPM as the single document to control exceptions or clarifications to a standard.

Entergy's QAPM is organized in a format that is consistent with NUREG-0800, Section 17.3, which lists RG 1.8, Revision 1, "Personnel Selection, and Training" as one of the regulatory guides to which applicants should comply or provide acceptable alternatives.

3.2.2 Personnel Training and Qualification

3.2.2.1 Training Requirements

The licensee has taken an exception to the training standards in Section 5 of ANSI/ANS 3.1-1978 in favor of the training requirements of 10 CFR Part 55 and 10 CFR 50.120. The regulations in 10 CFR Part 55 address licensed operator training. The regulations in 10 CFR 50.120 require licensees to provide training for specific categories of non-licensed plant personnel that is derived from a systems approach to training (SAT). However, the current QAPM does not commit to ANSI/ANS 3.1-1978, Section 5, and its omission does not represent a reduction in commitment, and is acceptable.

Entergy commits to ANSI/ANS 3.1-1978 Sections 1-4. Section 5 contains standards for a training program that was developed prior to the issuance of 10 CFR 50.120, "Training and qualification of nuclear power plant personnel," that established training requirements for all licensees. The regulations in 10 CFR 50.120 require licensees to establish, implement, and maintain a training program that meets specified requirements. The nuclear industry has developed training guidance through the Institute of Nuclear Power Operations (INPO), which Entergy is implementing to meet the requirements of 10 CFR 50.120. The training program must be developed to be in compliance with the facility license, including all TS and applicable regulations. These training programs are periodically evaluated and revised as appropriate to reflect industry experience as well as changes to the facility, procedures, regulations, and QA requirements.

Criterion II, "Quality Assurance Program," of Appendix B to 10 CFR Part 50 states, in part, that "The program shall provide for indoctrination and training of personnel performing activities affecting quality as necessary to assure that suitable proficiency is achieved and maintained." In its submittal, Entergy stated that the QAPM sentence is an administrative change that clarifies which sections of ANSI/ANS 3.1-1978 apply to the Entergy fleet and does not reduce any commitments in the QAPM or affect compliance with Appendix B, Criterion II.

3.2.2.2 Evaluation of Standardization Regarding Use of ANSI/ANS 3.1-1978

Entergy's QAPM qualification requirements for personnel, other than licensed operators covered under 10 CFR Part 55, currently meet ANSI/ANS 3.1-1978, except for positions where an exception to either ANSI/ANS 3.1-1978 or N18.1-1971, is stated in the applicable unit's TS. If an exception exists for a given position, the applicable unit's TS qualification requirements apply. The intent of the QAPM is to apply the qualification requirements of ANSI/ANS 3.1-1978 to all Entergy units except to the extent that exceptions are specifically stated in the QAPM or the TSs. The current wording of the QAPM is intended to ensure that each unit complies with both the QAPM and TSs regarding staff qualifications. In its submittal, Entergy proposed to clarify the scope of the 1978 standard to be applied to its units, clarify the exceptions, and eliminate TS disparities by revising both the QAPM and the TSs. Entergy proposed committing to Sections 1-4 of ANSI/ANS 3.1-1978 with the following clarifications and exceptions:

- The radiation protection manager shall meet or exceed the qualifications of Regulatory Guide 1.8, Revision 2, 1987.

- Entergy managers required to hold a Senior Reactor Operator (SRO) license are specified in the applicable unit's Technical Specifications.
- Entergy's QAPM will state that licensed operators shall be qualified in accordance with the requirements of 10 CFR Part 55.

The following new sentence is proposed for clarification:

Entergy is committed to Sections 1 - 4 of ANSI/ANS 3.1-1978 with the following clarifications and exceptions.

Currently, all of Entergy's units are committed to ANSI/ANS 3.1-1978, so the use of ANSI/ANS 3.1-1978 for qualification of the general personnel is not a change. However, the Entergy fleet is also committed to ANSI N18.1-1971. Entergy stated that this dual commitment has caused confusion in the past and requested to delete any commitment to and any exceptions from ANSI N18.1-1971 from the TSs and the QAPM. The ANSI/ANS 3.1-1978 standard was not officially endorsed by a regulatory guide, but this standard has been found acceptable by the NRC staff as discussed in Section 3.2.1 of this safety evaluation. In its submittal, Entergy proposed adoption of the ANSI/ANS 3.1-1978 standard, which is currently contained in the QAPM and plant TSs.

Since there are several precedents within the Entergy fleet in which the NRC approved replacing N18.1-1971 with ANSI/ANS 3.1-1978 (ANO-1 (ADAMS Accession No. ML013050554), ANO-2 (ADAMS Accession No. ML042720521), GGNS (ADAMS Accession No. ML031130220), IP2, and IP3 (ADAMS Accession No. ML071990307)), the NRC staff concludes that the replacing references to ANSI N18.1-1971 with ANSI/ANS 3.1-1978 in the Entergy QAPM and the fleet's administrative TSs is acceptable.

Based on the above, the NRC staff concludes that the proposed changes describe controls that, when properly implemented, will meet the guidance provided in NUREG-0800, Section 17.3, and the requirements of Appendix B to 10 CFR Part 50. Therefore, the NRC staff concludes that the proposed changes to standardize the Entergy commitments to the ANSI/ANS 3.1-1978 standard are acceptable.

3.2.2.2 Evaluation of RPM Qualifications

Entergy's QAPM changes propose that the RPM qualification requirements be relocated from the TS to the QAPM and revised to be consistent across the fleet. Currently, the individual plant TSs specify the qualification requirements for the RPM. The latest standard to which any plant is committed is RG 1.8, Revision 2, 1987. All other units are committed to RG 1.8, Revision 1, 1975. In its submittal, Entergy proposed to apply RG 1.8, Revision 2, 1987, to all Entergy plants. The relocated exception in the QAPM would state,

The radiation protection manager shall meet or exceed the qualifications of Regulatory Guide 1.8, Revision 2, 1987.

The educational requirements for RG 1.8, Revision 1, 1975 states that the RPM should have a bachelor's degree or the equivalent in a science or engineering subject, including some formal training in radiation protection. The educational requirements from RG 1.8, Revision 2, 1987, state that the RPM should have a bachelor's degree in a science or engineering subject, including formal training in radiation protection.

Although Revision 2 of RG 1.8 does not use the term "or equivalent" when referring to a bachelor's degree, section 4.1 of ANSI/ANS 3.1-1981 as endorsed by RG 1.8 provides an allowance for the RPM to be qualified without a bachelor's degree. The standard states, in part that

Individuals who do not possess the formal educational requirements specified in this section shall not be automatically eliminated where other factors provide sufficient demonstration of their abilities. These other factors shall be evaluated on a case by-case basis and approved and documented by the plant management.

The current revision of Entergy's QAPM contains an allowance to preclude disqualifying individuals who met previous commitments but do not meet the more restrictive new commitments. The QAPM currently states, in part, that

Individuals filling positions who met the previous commitment at the time of implementation of this commitment can be considered to meet any more restrictive aspects of the requirements of this commitment for that position without further review and documentation.

This allowance is retained in the QAPM and would apply to any individual currently serving in the RPM position that does not fully meet the updated guidance.

The guidance of RG 1.8, Revision 1, states that the RPM should have at least 5 years of professional experience in applied radiation protection, and at least 3 years of this professional experience should be in applied radiation protection work in a nuclear facility dealing with radiological problems similar to those encountered, preferably, in an actual nuclear power station. In comparison, RG 1.8, Revision 2, 1987, indicates that the RPM should have 4 years of experience in applied radiation protection, and 3 years of this experience should be in applied radiation protection work in a nuclear facility dealing with radiological problems similar to those encountered preferably in a nuclear power plant. The change is a reduction in commitment of total professional experience for the RPM from 5 years to 4 years.

The guidance of RG 1.8, Revision 1, 1975, states a master's degree is to be considered equivalent to 1 year of professional experience, and a doctor's degree may be considered equivalent to 2 years of professional experience where course work related to radiation protection is involved. This allowance is not contained in RG 1.8, Revisions 2, 1987, or the endorsed ANSI standard. ANSI/ANS 3.1-1981 includes additional standards for the 3 years experience at a nuclear facility. The ANSI standard indicates that during the 3 years, the individual should participate in the radiation protection section of an operating nuclear power plant during the following periods: routine refueling outage for up to 2 months, and 2 months

operation above 20 percent power. Six months experience should be onsite. In addition to this guidance, ANSI/ANS 3.1-1981 includes new standards for the individual who temporarily replaces the radiation protection group leader. The ANSI standard indicates that the replacement should have a bachelor's degree in a science or engineering subject and 2 years experience, one of which should be nuclear power plant experience. As noted in regards to education above, alternatives to the bachelor's degree may also be applied to a temporary replacement.

There is a reduction in the total minimum years of experience (from 5 years to 4 years); but the reduction is offset by more specific commitments on the type of experience. The 4 years experience should include 3 years of radiation protection professional-level experience. The individual should also have participated in the radiation protection section of an operating nuclear power plant during a routine refueling outage (1 to 2 months) and 2 months above 20 percent power. The individual should also have 6 months of on-site experience. Accordingly, the proposed change to standardize and/or upgrade in experience level in a specific area, outweighs the small reductions for years of experience in Entergy plants.

Based on the above, the NRC staff concludes that the proposed QAPM change, when properly implemented, will meet the guidance in SRP 17.3, and therefore, will meet the requirements of Appendix B to 10 CFR Part 50.

Therefore, the NRC staff concludes that the proposed change to the RPM qualifications is acceptable.

3.2.2.3 Evaluation of Managers Required to Hold an SRO License

Section 4.2.2 of ANSI/ANS 3.1-1978, which was accepted by the NRC in its safety evaluation dated November 6, 1998, indicates that at the time of initial core loading or appointment to the position, whichever is later, the operations manager should hold an SRO's license. The Entergy plant TSs take exception to this provision, specifying that either the operations manager or an operations middle manager (or assistant operations manager) shall hold an SRO license. The current QAPM proposed revision plans to delete the general reference to exceptions specified in the TS to avoid confusion. Therefore, the exception for this provision is specifically stated in the QAPM as, "Qualification requirements for personnel shall meet ANSI/ANS 3.1-1978 except the following," and, "Managers required to hold an SRO license are specified in the applicable unit's Technical Specifications."

The NRC staff concludes that the change is administrative in nature and has no impact on current commitments, but is included here for completeness of the overall requested change, and, therefore, finds it acceptable.

3.2.2.4 Evaluation of Licensed Operators

Entergy's QAPM currently states, "Qualification requirements for personnel, other than Licensed Operators covered under 10CFR55, shall meet ANSI/ANS 3.1-1978." Entergy's exception for licensed operators is reworded and relocated to the list of exceptions as follows, "Qualification requirements for personnel shall meet ANSI/ANS 3.1-1978 except the following...

- c. Licensed Operators shall be qualified in accordance with the requirements of 10 CFR 55.

This change is administrative and does not alter existing commitments. Licensed operators will continue to be qualified in accordance with 10 CFR Part 55. This exception remains consistent with the NRC staff's safety evaluation dated November 6, 1998, that consolidated Entergy's QAPM, which states, "In addition, more restrictive requirements are still in place in Technical Specifications, and other regulations where appropriate (e.g., licensed operator qualifications)."

The NRC staff concludes that the change is administrative in nature and has no impact on current commitments, but is included here for completeness of the overall requested change, and, therefore, finds it acceptable.

3.2.2.5 Evaluation of Functional Titles

In its submittal, Entergy proposed to add Insert 2, below, to clarify the different functional titles and the titles reference in ANSI/ANS 3.1-1978, Section 4.4. Entergy's proposed clarification, regardless of the title, ensures the position has the authority and specified qualifications to accomplish the functional responsibilities.

Individuals assigned to professional-technical comparable positions shall have the authority and specified qualifications to accomplish the functional responsibilities of the position.

ANSI/ANS 3.1-1978 provides qualification standards for specified functional areas of responsibility for nuclear organizations. For differences between functional level titles used in the ANSI standard and those used by Entergy organizations, Entergy proposed the above clarification to ensure that the comparable position has the authority, and specified qualifications to accomplish the functional responsibilities.

The NRC staff concludes that the change is administrative in nature and has no impact on current commitments, but is included here for completeness of the overall requested change, and, therefore, finds it acceptable.

3.2.2.6 Shift Technical Advisor (STA)

For STA qualification, Entergy proposed to commit all plants in its fleet to the latest standard, ANSI/ANS 3.1-1993 as endorsed by RG 1.8, Revision 3, 2000, including a clarification as to which plant MODES the STA must be on-shift serving in the STA role.

NUREG-0737, "Clarification of TMI Action Plan Requirements," November 1980 (ADAMS Accession No. ML051400209), states, in part, that "... an STA (shall) be available for duty on each operating shift when a plant is being operated in Modes 1-4 for a PWR and Modes 1-3 for a BWR. At other times, an STA is not required to be on duty." The TS wording proposed by the licensee includes plant-specific terminology that differs slightly from the NUREG-0737 wording, but accomplishes the same purpose. Accordingly, the NRC staff concludes that the licensee's commitment to RG 1.8, Revision 3, is acceptable.

3.2.2.7 Evaluation of Quality Assurance Manager Qualifications

Section 4.4.5 of ANSI/ANS 3.1-1978 prescribes the following qualifications for the person responsible for Quality Assurance:

At the time of initial core loading or assignment to the active position, the responsible person shall have six years experience in the field of quality assurance, preferably at an operating nuclear plant, or operations supervisory experience. At least one year of this six years experience shall be nuclear power plant experience in the overall implementation of the quality assurance program. (This experience shall be obtained within the quality assurance organization). A minimum of one year of this six years experience shall be related technical or academic training. A maximum of four years of this six years experience may be fulfilled by related technical or academic training

Neither the current QAPM nor ANSI/ANS 3.1-1978, approved by the NRC staff's safety evaluation dated November 6, 1998, provides consideration for candidates for the position that do not fully meet all of the above criteria, but; based upon other factors, may be an appropriate choice for the position. However, the NRC staff guidance in NUREG-0800, Section 17.5, includes this allowance. Entergy's revision to the QAPM would include this allowance as Insert 3 below:

Individuals who do not possess the formal education and minimum experience requirements for the manager responsible for quality assurance should not be eliminated automatically when other factors provide sufficient demonstration of their abilities. These other factors are evaluated on a case-by-case basis, approved, and documented by senior management.

As a minimum, the Special Requirements of ANSI/ANS 3.1-1993 Section 4.3.7 should be met if the manager responsible for QA does not meet the standards of Section 4.4.5 of ANSI/ANS 3.1-1978.

In its submittal, Entergy proposed the following new exception to ANSI/ANS 3.1-1978 Section 4.4.5:

Individuals who do not possess the formal education and minimum experience requirements for the manager responsible for quality assurance should not be eliminated automatically when other factors provide sufficient demonstration of

their abilities. These other factor are evaluated on a case-by-case basis and approved and documented by senior management

Entergy requested to apply the NUREG-0800, Section 17.5 allowance by adding it to the QAPM as an exception to Section 4.4.5 of ANSI/ANS 3.1-1978. This change is a reduction in commitment. Nonetheless, an individual being considered for the position will be evaluated on a case-by-case basis as to their capabilities to perform the duties and responsibilities of the position. Use of the exception would be approved by senior management and documented. Without the exception, some candidates that are fully capable of fulfilling the duties and responsibilities would be automatically excluded from consideration. Appendix B to 10 CFR Part 50 does not specify particular qualification requirements, but requires that an organization be capable of fulfilling the duties and responsibilities specified in the QA Program. The proposed change continues to ensure that the QA manager is sufficiently qualified to fulfill the duties and responsibilities of the position.

In view of the above, the proposed change describes controls that, when properly implemented, will meet the guidance or SRP, and, therefore, meet the requirements of 10 CFR Part 50, Appendix B. The QAPM does not leave the qualifications open, but preserves minimum qualification standards by specifying, "Special Requirements of ANSI/ANS 3.1-1993 Section 4.3.7 must be met if the manager responsible for Quality Assurance does not meet the requirements of section 4.4.5 of ANSI/ANS 3.1-1978."

3.2.2.7 Evaluation of Training for Managers, Supervisors, and General Personnel

The current QAPM addresses qualification commitments; it does not clearly specify training program commitments. Entergy has proposed a change to provide a single point of reference for a Training Program commitment in the QAPM to train applicable licensee staff identified in the ANSI standard that are not covered by 10 CFR 50.120 or 10 CFR Part 55.

In its submittal, Entergy proposed to modify the proposed QAPM exception to Section 5 of ANSI/ANS 3.1-1978 to specifically address training commitments with the proposed QAPM Insert 4 as follows:

Entergy will maintain a training program for the unit staff that meets the applicable regulations and either a) is accredited by the National Nuclear Accrediting Board (NNAB) or b) meets the standards of section 5 of ANSI/ANS 3.1-1978.

Proposed QAPM Insert 4 allows certain training programs to be accredited by the National Nuclear Accrediting Board (NNAB) as part of the training accreditation program developed by the INPO in lieu of meeting Section 5 of ANSI/ANS 3.1-1978. Typical accredited training programs include training for licensed and non-licensed operators (initial and requalification programs), professional technical personnel (engineering support personnel), technicians, and maintenance personnel. Entergy states that programs which are not accredited by the NNAB, such as general employee training and supervisor training, will continue to meet the standards of Section 5 of ANSI/ANS 3.1-1978.

The above change describes controls that, when properly implemented, will meet the guidance in NUREG-0800, Section 17.3 and, therefore, will meet the requirements of 10 CFR Part 50, Appendix B. The proposed upgrade from the current standard of ANSI/ANS 3.1-1978 to the more rigorous NNAB accreditation is an enhancement and an upgrade in commitment. Therefore, the NRC staff concludes that the above change to training for managers, supervisors, and general personnel is acceptable.

3.2.2.8 "Grandfathering" without Justification and Documentation

Because the only personnel who will be affected by this exception are already qualified incumbents who are performing their jobs on a daily basis, confirming and documenting that they satisfy the education and/or experience requirements for their positions would serve no purpose, and could represent a significant administrative burden for the licensee. When the incumbents leave their positions, new candidates will be judged in accordance with the proposed qualification standards. According to the licensee, implementation of this "Grandfather" clause will necessitate changes to fleet procedure EN-HR-137, "Complying with the Standards for Selecting Nuclear Power Plant Personnel," to ensure that replacements of incumbents meet the new requirements prior to employment in the position. Based on the lack of value added, the staff finds that re-documenting the qualifications of qualified incumbents is not necessary. Therefore, the NRC staff concludes that the "grandfathering" clause proposed by Entergy is acceptable.

3.2.2.9 Relocation of Qualification Requirements and Exceptions from TSs to the QAPM

Allowing exceptions to qualification requirements to be documented in the QAPM instead of TSs is less restrictive, but justifiable in terms of workload efficiency for both the licensee and the NRC staff. The relocation of the exceptions from TSs to the QAPM is acceptable because 10 CFR 50.54(a)(4) provides adequate control over changes to the exceptions in the QAPM. Specifically, § 50.54(a)(4) requires prior NRC approval of (1) a change to all or any portion of an exception if the change would delete or otherwise reduce one or more of the qualifications called for by the exception, and (2) addition or modification of any exception that would allow for fewer qualifications than those called for by ANSI/ANS 3.1-1978. Further, a change to the QAPM that adds qualifications, whether in an existing exception in the QAPM or a new one, will not impair the existing provisions regarding qualification, and the NRC need not approve such changes through the license amendment process. Accordingly, the NRC staff concludes that this change is acceptable.

3.2.3 Summary

Based on the above evaluation, the NRC staff concludes that the Entergy QA program described in the revised QAPM satisfies the Commission's requirements for QA programs as established by Appendix B to 10 CFR Part 50. The program description adequately describes how the requirements of Appendix B will be implemented. Therefore, the staff concludes that the proposed Entergy QAPM changes continue to meet the 10 CFR Part 50 requirements for the QA program.

4.0 STATE CONSULTATIONS

In accordance with the Commission's regulations, the State officials for Arkansas, Mississippi, Louisiana, New York, Massachusetts, Vermont, and Michigan were notified of the proposed issuance of the amendments. The State officials had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: G. Lapinsky, A. Armstrong

Date: December 28, 2012

- Amendment No. 178 to Facility Operating License No. NPF-47 for River Bend Station, Unit 1;
- Amendment No. 253 to Renewed Facility Operating License DPR-28 for Vermont Yankee Nuclear Power Station; and
- Amendment No. 240 to Facility Operating License No. NPF-38 for Waterford Steam Electric Station, Unit 3.

The amendment changes the Entergy Quality Assurance Program Manual and Associated Plant TSs regarding staff qualifications. The changes standardize unit staff qualification requirements for the Entergy fleet.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,
/RA/

N. Kalyanam, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-313, 50-368, 72-13
50-155, 72-43, 50-333, 72-12, 50-416,
72-50, 50-003, 50-247, 50-286, 72-51,
50-255, 72-7, 50-293, 50-458, 72-49,
50-271, 72-59, 50-382, and 72-75

Enclosures:

1. Amendment No. 248 to DPR-51
2. Amendment No. 296 to NPF-6
3. Amendment No. 304 to NPF-59
4. Amendment No. 193 to NPF-29
5. Amendment No. 271 to NPF-26
6. Amendment No. 248 to NPF-64
7. Amendment No. 249 to DPR-20
8. Amendment No. 239 to DPR-35
9. Amendment No. 178 to DPR-47
10. Amendment No. 253 to DPR-28
11. Amendment No. 240 to DPR-38
12. Safety Evaluation

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